



Corrigendum: A novel method of studying the core boron transport at ASDEX Upgrade (2018 *Plasma Phys. and Control. Fusion* **60** 085011)

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1. Corrigendum

The shape of the experimental boron density profile that appears in our paper (figures 2, 3(a) and 12) is incorrect. As a result, the comparisons to the theoretical results presented in section 4 (figures 13 and 15) are also incorrect. The agreement between the experimentally measured and theoretically predicted impurity particle convection is significantly better than was reported in our publication.

There were two errors that contributed to the incorrect boron density profile. The first was an error in the absolute intensity calibration of the charge exchange recombination spectroscopy (CXRS) diagnostic used for the analysis [1]. This error was corrected by cross-calibrating to a second system [2] and by using the post-campaign (opposed to the pre-campaign) absolute-intensity calibration measurements. The second error was in the implementation of the ‘COLRAD’ package within the CHICA code [3] that was used to calculate the neutral density populations. The error resulted in incorrect distributions of halo neutrals, which lead to artificially hollow density profiles. This implementation error has since been corrected. Please note that these errors do not affect any other publication. In particular, the authors would like to stress that the impurity density profiles used in previous transport studies [4–7] do not suffer from either of these errors as they were calculated using data from the NBI I CXRS system only [2] and using the ‘FIDASIM’ code rather than ‘COLRAD’ for the neutral distributions.

With these errors corrected, the boron density profiles in this discharge are flat, not hollow. The boron density profiles from both CXRS systems are shown in figure 1. The shapes of the profiles agree well within the error bars. Here, the boron density profile from the NBI Box II system has been scaled upward to match the NBI Box I system. The discrepancy in the absolute magnitudes is the result of significant damage to the optical heads that occurred during the experimental campaign.

Figure 3 from the original publication, which showed the temporal evolution of the measured modulation data and its simulated counterpart, is reproduced here (figure 2) using the correct boron density profiles. Here, the data from all CXRS diagnostics have been included while in the original publication only data from the NBI Box II system were used. The authors felt it important to include the data from NBI Box I system, as the profile shape measured on this system is more reliable.

The numerical scheme and methodology presented in the original publication are still valid. However, the results and comparisons presented in section 4 that are connected to the shape of the density profile are not. The new modulation analysis for this discharge together with the results of the minimisation are shown in figure 3. The phase profile is the same as in the original data and the amplitude profile is quite similar within its uncertainties, see figure 12 (original publication) for comparison. Here, the amplitude profile is shown in units of density rather than in percent as was done

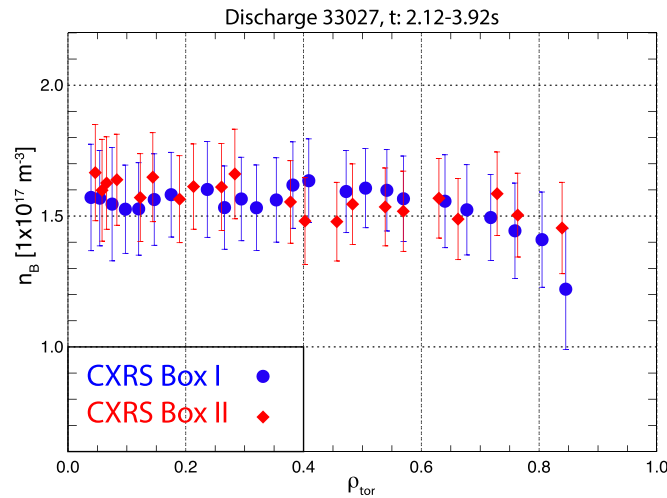


Figure 1. Steady-state boron density profiles from the two CXRS core diagnostics in discharge 33 027. These profiles have been averaged over the time window 2.12–2.92 s. The data from the NBI Box II system have been scaled upward by a factor of 1.6 to bring them into agreement with the profile from the NBI Box I diagnostic.

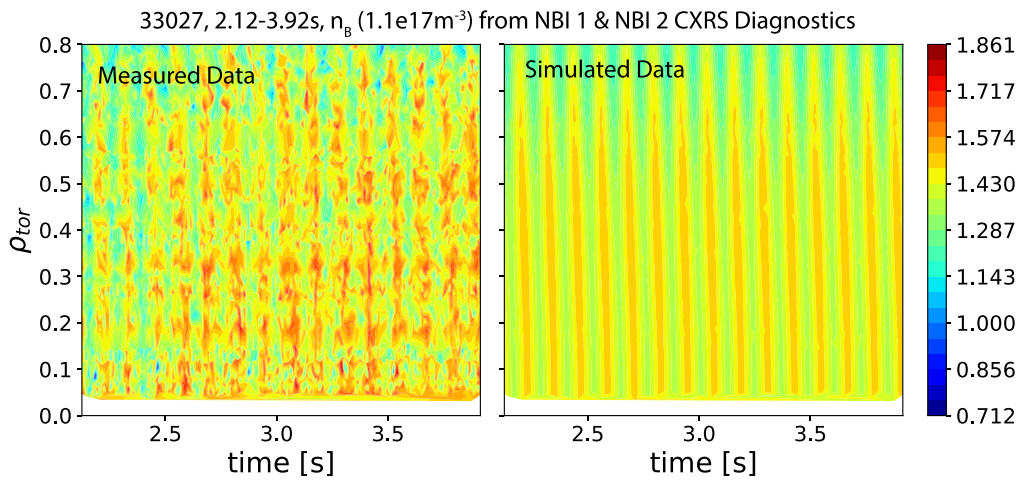


Figure 2. Figure 3 from original publication using correct boron density profiles. Left: contour plot of the measured boron density. Right: contour plot of the reconstructed boron density. Note that the two plots have the same colorbar.

in the original publication. The major change is the steady-state profile, which sets the ratio of diffusion to convection. The data in figure 3 appear noisier than in the original publication. The uncertainties, however, are extremely similar. The appearance of increased ‘noisiness’ results from significantly more data in closer radial proximity and the much flatter profile gradient. As in the original publication, the blue lines are the result of the best minimization to the dataset.

The minimization using the corrected boron density profile also yields new corrected diffusion and convection profiles. These can be seen in figure 4, which is a reproduction of figure 13 from the original publication with updated experimental data. The thick blue lines correspond to the minimization shown in figure 3. In addition, light blue uncertainty intervals are shown. These intervals correspond to the range of D and V values obtained from minimizations performed on variations of the original dataset altered to explore the maximum uncertainty ranges of the measurements. In particular, the

gradients of the steady-state, phase, and amplitude profiles were varied by $\pm\sim 10\%$ to cover the steepest and shallowest profile gradients supported by the measurements. The minimizations corresponding to the variation of the steady-state profile are shown as dashed lines in figure 3. The blue shaded uncertainty interval can be thought of as a maximum error bar.

The new diffusion profile is similar to the original, but slightly larger giving even better agreement with the combined predictions from GKW and NEO. The most important difference is in the convective profile shown on the right hand side of figure 4. The boron density profiles in the original publication were hollow, leading to outward convection and poor agreement with the theoretical predictions. Here, the convection is found to be inward around mid-radius, leading to very good agreement with the predictions, even quantitatively so within the uncertainties of the measurements. For this plasma, the linear (and non-linear) predictions by GKW and NEO are able to accurately reproduce the experimental data.

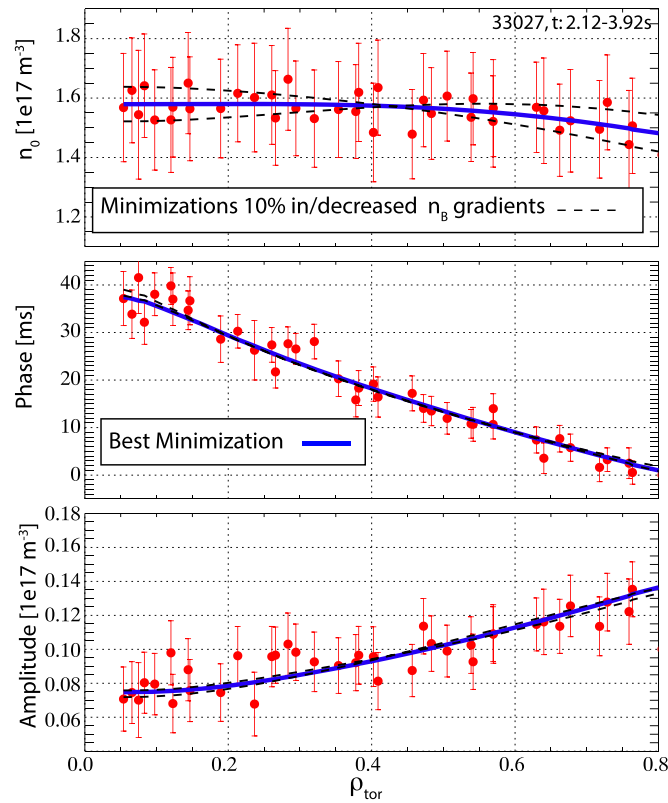


Figure 3. Figure 12 from original publication using correct boron density profiles. Steady-state, phase, and amplitude of the measured data (red) and the simulation (blue). The blue lines are not fits to the red data points, but rather the results of the minimization. The black dashed lines correspond to minimizations performed on datasets that were altered to have 10% increased and decreased gradients of the steady-state profile.

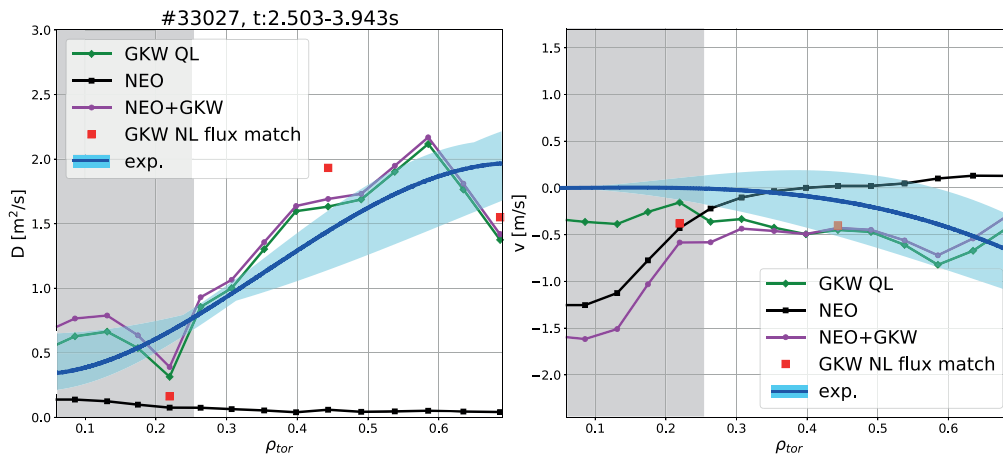


Figure 4. Reproduction of figure 13 from the original publication using new corrected boron density profiles. Experimental and theoretical D (left) and v (right) profiles. The experimental profiles with their uncertainty bands are depicted in blue, the ones from the quasi-linear GKW run in green diamonds, from NEO in black squares, and the total theoretical profiles in magenta circles. The red squares represent the result of the nonlinear GKW run performed with matching of the heat fluxes. Our region of interest starts at $\rho_{tor} = 0.25$, hence, where the gray region ends.

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References

- [1] McDermott R M *et al* 2017 *Rev. Sci. Instrum.* **88** 073508
- [2] Viezzer E *et al* 2012 *Rev. Sci. Instrum.* **83** 103501
- [3] McDermott R M *et al* 2018 *Plasma Phys. Contro. Fusion* **60** 095007
- [4] Casson F J *et al* 2013 *Nucl. Fusion* **53** 063026
- [5] Angioni C *et al* 2011 *Nucl. Fusion* **51** 023006
- [6] Kappatou A *et al* 2018 *Plasma Phys. Contro. Fusion* **60** 055006
- [7] Kappatou A *et al* 2018 *Nucl. Fusion* **59** 056014

A novel method of studying the core boron transport at ASDEX Upgrade

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Abstract

A new ion cyclotron resonance frequency (ICRF) modulation technique has been developed and exploited at ASDEX Upgrade for obtaining time perturbed boron density signals. Square wave modulation of the ICRF heating power results in a periodic modulation of the boron density in the edge, which propagates inward. From this time perturbed boron density signal, the boron diffusivity and convection in the radial transport equation are individually determined. This task is done by solving an inverse problem by a quasi-Newton method. This implemented framework has been verified with the Method of Manufactured Solutions and benchmarked with the transport code STRAHL. The experimental transport coefficients are compared with neoclassical calculations with the code NEO and gyrokinetic simulations with the code GKW. The neoclassical diffusivity is well below the experimental one. The comparison to gyrokinetic theory shows a good agreement in the diffusivity, but the theoretical convection predicts a more peaked boron density profile than measured in the experiment.

Keywords: tokamak, modulation experiment, CXRS, neoclassical and turbulent transport, ICRF modulation, H-mode, light impurity transport

(Some figures may appear in colour only in the online journal)

1. Introduction

Impurities in fusion plasmas arise from many different sources including the erosion and sputtering of materials from plasma facing components, the intentional injection of impurities via gas puffing, pellets, or laser blow-offs for divertor cooling, core radiation control, or physics studies, and the production of helium from the fusion process itself. The plasma performance is highly affected by the impurity concentration and, to achieve a stable burning plasma scenario in future reactors, the build up of impurities in the plasma core must be controlled. Therefore, a fundamental

understanding of the mechanisms behind impurity transport in fusion plasmas as well as identifying the engineering parameters with which the transport can be controlled are of great importance. In the past years impurity transport studies of both low-Z and high-Z impurities have been conducted in a broad variety of plasma conditions in several different tokamaks around the world. The most common ways of injecting impurities for transport studies are with gas puffing [1–5] or laser blow-offs [6–9].

Impurity transport perpendicular to flux surfaces can be described by diffusion and convection. This can be formulated in the radial transport equation [10]

$$\frac{\partial n_Z(r, t)}{\partial t} = \frac{1}{r} \frac{\partial}{\partial r} r \left(D(r) \frac{\partial n_Z(r, t)}{\partial r} - v(r) n_Z(r, t) \right),$$

$$n_Z(r_{\max}, t) = s(t). \quad (1)$$



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In equation (1), $n_Z(r, t)$ is the impurity density for an impurity species Z , $D(r)$ is the diffusion coefficient, $v(r)$ is the convection or drift velocity, and $s(t)$ is the impurity density boundary condition at the edge, r_{\max} . This equation is written in cylindrical coordinates, with r being the radial coordinate, which has been chosen such that $r = \sqrt{V/(2\pi^2 R_{\text{axis}})}$, where V is the volume within the flux surface and R_{axis} is the major radius of the magnetic axis [11]. With this choice the flux surfaces are transformed into circles conserving the plasma volume [11]. Note that in equation (1) the transport coefficients have no dependence on the poloidal angle and, therefore, represent flux surface averaged quantities [12]. In tokamak plasmas, magnetic flux surfaces are in general not circular, but are approximately elliptic. This ellipticity is neglected in the analysis. However, it is possible to estimate the impact of the ellipticity on the deduced transport coefficients by calculating the missing factor of $\langle \nabla r^2 \rangle$ [13]. Including this factor gives a modification to the particle flux and, thus, to the transport coefficients of 2%–3% in the center, 7%–8% at mid-radius, and up to 20% at $\rho_{\text{tor}} \sim 0.7$. These small corrections are well within the experimental error bars.

If the impurity density profile is constant in time, that is $\frac{\partial n_Z(r, t)}{\partial t} = 0$, equation (1) reduces to the steady-state equation

$$\frac{1}{n_Z} \frac{\partial n_Z}{\partial r} = \frac{v}{D}. \quad (2)$$

Suppose a D and v are found that satisfy equation (2). Then it is straight-forward to see that $\tilde{v} = Cv$ and $\tilde{D} = CD$, where C is a positive constant, is also a solution to equation (2). Hence, from only the steady-state equation (2), the transport coefficients cannot be uniquely determined. To disentangle v and D from one another, one needs to measure the temporal evolution of the impurity density profiles after a perturbation, e.g. during a modulation of the impurity source at the plasma edge.

In this work, a novel method of inducing a time perturbed boron density at ASDEX Upgrade (AUG) has been developed and utilized to uniquely determine the transport coefficients D and v in AUG H-mode plasmas. This method combines the advantages of a transient transport analysis performed on modulated signals over many periods, as often applied for heat transport studies, with the high radial resolution enabled by charge exchange recombination spectroscopy (CXRS), which, at AUG has so far only been used for the measurement of impurity density profiles for transport analysis in stationary conditions [10, 14]. The use of CXRS limits the applicability of the method to sufficiently light impurities. This, however, does not reduce the physics interest of this approach, which, as it will be shown in the present application, delivers accurate, highly radially resolved, separated measurements of the boron transport coefficients. The importance of determining the diffusion D and drift velocity v separately can be readily understood considering that this also allows more complete comparisons with the theoretical predictions of impurity transport models. This lends itself directly to the goal of increasing our confidence in the prediction of the diffusion of helium, which is a critical parameter in determining the impact of the central helium source produced by fusion reactions. Moreover, accurate investigations dedicated to light

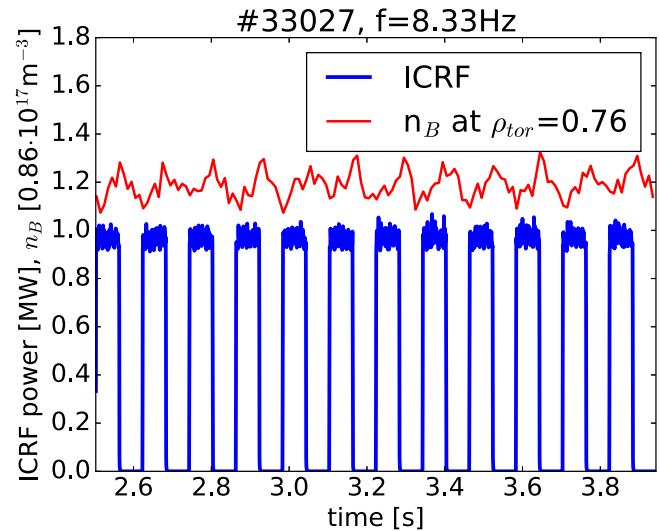


Figure 1. ICRF heating power modulation with a frequency of 8.33 Hz (blue) and the resultant modulation of the boron density at $\rho_{\text{tor}} = 0.76$ (red).

impurities are of critical importance in the validation of impurity turbulent transport models. This is an important element in the prediction of the impurity density profiles in a fusion reactor plasma, also for highly charged impurities. In fact, in present experiments the transport of heavy impurities is much more dominated by the neoclassical convection [15], in contrast to light impurities which are more dominated by turbulent transport [2, 3, 5, 14]. At the very low collisionalities of a reactor, however, the role of turbulent transport can be expected to become significantly more important also for heavy impurities [16].

This paper is organized as follows: section 2 describes the new modulation method as well as how the transport coefficients can be deduced from this modulation signal. In section 3 the validation of the method as well as its uncertainties are discussed. The first experimental results with an initial comparison to neoclassical and gyrokinetic theory are presented in section 4. This is followed by a conclusion in section 5.

2. Methodology

2.1. Boron density modulation with ICRF

The experiments in this work were conducted at AUG, which is a medium-sized, tungsten walled, divertor tokamak situated at the Max-Planck Institute for Plasma Physics in Garching, Germany [17]. The major and minor radii of the machine are 1.65 m and 0.5 m, respectively, and it is equipped with a comprehensive suite of diagnostics as well as flexible and powerful heating systems including neutral beam injection (NBI), electron cyclotron resonance heating (ECRH), and ion cyclotron resonance frequency (ICRF) heating [18]. At AUG, it was observed that a modulation of the power of the boron- and tungsten-coated ICRF antennas results in a modulation of the boron density in the plasma. This can be clearly seen in figure 1, where the ICRF heating power is presented in blue

and the resultant boron modulation signal in red for one CXRS channel at $\rho_{\text{tor}} = 0.76$. The boron modulation is stronger at the edge than in the core and the edge also responds more quickly than the core to the change in the ICRF heating power, which indicates a change in the boron source from the edge plasma rather than a change in core transport, as the measurement of the boron influx at the limiters also increases when the power of the antennas is modulated. Therefore, this technique can be used for core boron transport studies. The exploration of this possibility is the subject of this work.

It was discovered already in the 1980s that the operation of ICRF antennas can cause an increased influx of metallic impurities [19, 20], which arises due to the RF sheath potentials in the Faraday screen gaps [21, 22]. The ICRF concept in this work is similar to the work in [23], in which the tungsten transport at the edge was studied by modulating the power of the ICRF antennas. The ICRF heating system at AUG has been updated and is now comprised of four antennas in pairs of two: two 2-strap boron-coated and two 3-strap tungsten-coated antennas [24]. In this study, all the ICRF antennas are operated in phase and the ICRF heating scheme used is the hydrogen minority heating with a frequency of the ICRF generators of 36.5 MHz.

A feasibility study was conducted to better understand the boron source and if the technique can be used for transport studies. At AUG, boron is considered to be an intrinsic impurity due to the regularly performed boronizations, during which the vessel wall is covered with a thin layer of boron [25, 26]. It has been observed that the boron modulation signal is strongest when the experiments are performed in a freshly boronized machine. This suggests that the ICRF heating power modulation affects the boron coating on the antennas which originates from the boronization and not the boron of the antenna itself. This hypothesis is strengthened by the fact that a strong modulation signal is also achieved when only modulating the tungsten-coated ICRF antennas. Furthermore, only a modulation of the ICRF antennas have an impact on the boron density, since attempts at modulating the ECRH at various power levels did not result in a boron modulation. To enable boron CXRS measurements of good quality at AUG, a minimum of 2.5 MW of NBI power is necessary when performing these experiments. The feasibility study also demonstrated that this technique is applicable in a broad variety of H-mode, lower single null plasma conditions, although only one experimental dataset is shown in this paper. Apart from that the method is insensitive to the current, magnetic field, and plasma parameters. So far, the ICRF modulation has only been observed to affect the boron content in the plasma; helium, carbon, and nitrogen show no modulation behavior. On the other hand, the concentrations of these impurities were very low, making a possible modulation perhaps beyond detection. All in all, this method may be applicable in other machines which have carbon walls or also utilize boronization as a wall conditioning technique.

A requirement for the feasibility of this technique is a steady plasma background, which means keeping the electron density, the ion, and the electron temperatures constant such

that the transport coefficients D and ν are not time-dependent during the modulation. From the feasibility experiments it is clear that the amplitude of the measured boron density modulation scales with the ICRF heating power, but increasing the ICRF heating power also causes a bigger modulation of the ion and electron temperatures. Therefore, one has to choose an ICRF power level at which a clear modulation of the boron density is observed, while the modulations of the other quantities are kept as low as possible. From the study it was concluded that a power level of ~ 1 MW of ICRF heating is sufficient to modulate the boron density up to 10% at the edge while keeping the modulation of the ion and electron temperatures to less than 1%–4%. Different modulation frequencies were tested and also here a balance between a clear boron modulation signal and the perturbation to the plasma background has to be maintained. The modulation frequency chosen for the ICRF heating power is 8–10 Hz and this frequency is directly translated to the perturbation seen in the boron density, which is clearly visible in figure 1.

2.2. Measuring the boron density with CXRS

At AUG two core CXRS systems regularly measure the boron content in the plasma. In total these systems have 72 lines-of-sight (LOS) and a typical integration time of 10 ms [27, 28]. The two core systems are located on opposite sides of the torus and, hence, view two different NBI sources, with beam energies of 60 keV (box I) and 93 keV (box II), respectively. The spatial and maximal temporal resolution of the system utilizing the 60 keV beam are 1.0–2.5 cm and 3.5 ms [27], while for the system utilizing the 93 keV beam they are 1–1.5 cm and 2.5 ms [28]. However, the systems are often signal limited to longer integration times. The experimental data shown in this paper is from the core system viewing the 93 keV beam (box II), but as a consistency check the density profiles are cross-checked and validated against the other core system viewing the 60 keV beam. The measurements presented in this paper were performed with an integration time of 10 ms.

The CXRS diagnostics measure the spectral line emission from the charge exchange interaction of the the neutral beam particles with the impurities in the plasma [29, 30]. The plasma rotation can be deduced from the measured Doppler shift and the ion temperature from the Doppler broadening of the spectral line. From the measured line intensities $I_{\text{CX,Z}}$ the impurity density n_z can be calculated in the following way:

$$n_z = \frac{4\pi}{h\nu} \frac{I_{\text{CX,Z}}(\lambda)}{\int_{\text{LOS}} \sum_i \sum_j \langle \sigma v \rangle_{i,j} n_{0,i,j} dl}, \quad (3)$$

where $\langle \sigma v \rangle_{i,j}$ is the effective CX emission rate coefficient for a given spectral line, $n_{0,i,j}$ are the neutral atoms from the beam for a given energy component i and a given excited state j of the beam. The calculation of the impurity density has been performed with the in-house CHICA code, which stands for charge exchange impurity concentration analysis. To calculate the impurity density, CHICA takes the electron density, electron temperature, ion temperature, the measured CXRS line intensity, and the plasma equilibrium as inputs. The

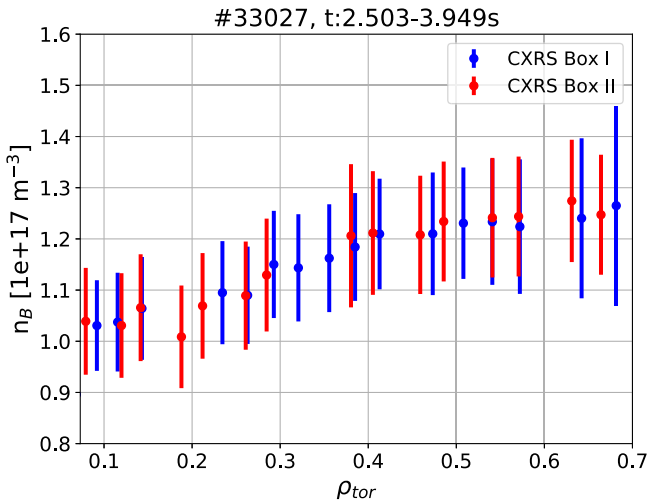


Figure 2. Steady-state boron density profile for the two CXRS core diagnostics. A correction factor of 0.88 has been applied to the NBI box I CXRS system to match the absolute value of the boron density profiles.

resultant impurity density profile depends strongly on these input profiles, especially the electron density, hence they have to be well known. Another important factor for the calculations is the geometry of the beams. The beam geometries have been constrained using beam emission spectroscopy (BES) measurements with different LOS as well as thermal images of the beam impact points on the inner wall. The cross-check of the calculated impurity density profiles between the CXRS systems on box I and box II, which have different geometries and energy components, provides confirmation that the beam geometries are correctly calculated. Differences in absolute magnitude of 10%–20% are often seen between the diagnostics and are attributed to calibration uncertainties and unavoidable deterioration of the in-vessel optical components during the experimental campaign. However, for determining the transport coefficients it is the gradient, e.g. the profile shape, that is important and, as can be seen in figure 2 and [28], both systems reproduce the same profile shapes. Note that in figure 2, a correction factor of 0.88 was applied to the density profile of the NBI box I CXRS system to match the absolute value of the impurity density profiles. Linear fits of the two datasets in the radial window $\rho_{\text{tor}} = 0.4\text{--}0.6$ are performed and the correction factor is determined from the ratio of the radial averages of the two linear fits. This is a standard method for combining the two datasets.

The neutral beam density n_0 in equation (3) can be measured with BES or calculated with various beam attenuation codes. In this work, a beam attenuation code has been utilized. The beam has three different energy components as well as a beam halo making $i = 1, 2, 3, 4$ in equation (3). Furthermore, the beam and the beam halo can be in different excited states j . The beam halo is a cloud of deuterium neutrals around the injected neutral beam, which are produced from charge exchange interactions between the injected neutral particles and the thermal plasma deuterium ions. The effective charge exchange emission rates $\langle\sigma v\rangle$ for a

given spectral line are derived from the charge exchange cross-sections found in the ADAS database [31]. The effective rate $\langle\sigma v\rangle$ is obtained by integrating σv over the velocity space of the beam neutrals and the impurity. The product of the neutral beam density n_0 and effective charge exchange emission rate $\langle\sigma v\rangle$ is integrated along the LOS and then summed over the different energy components and excited states. Note that in equation (3) the boron density is assumed to be constant along the intersection of the LOS and the neutral beam volume. This assumption is well met in the plasma core where the gradient lengths are small, but is not valid for the CXRS LOS that intersect in the edge pedestal inside of the beam volume. This is the case for the outermost LOS of both core systems ($\rho_{\text{tor}} > 0.85$). For these LOS, special analysis is required that has not been performed here and to avoid any effect of uncertain gradient on the profiles, the analysis region is restricted to $\rho_{\text{tor}} < 0.70$.

2.3. Numerical scheme

Solving equation (1) for the impurity density n_Z given a D and v is called the *forward problem*. However, the situation at hand is the opposite: the measured boron density n_B is known and the corresponding D and v profiles should be deduced. This is the *inverse problem*. This task, unlike the simple forward problem, is non-trivial since the problem itself is ill-posed, thus small errors in the measured data are greatly amplified in the solution. One way of regularizing the problem is to impose smooth D and v profiles. The boron density n_B is measured at discrete radial locations $(r_l)_{l=1,\dots,N_{\text{los}}}$ and time points $(t_k)_{k=1,\dots,N_t}$, where r_l denotes the radial and t_k the time measurement positions. t_k depend on the integration time of the CXRS diagnostic and r_l on the geometry of the LOSs and their intersection volumes with the NBI. This yields the measured data points $n_B^{l,k} = n_B(r_l, t_k)$, such that there is no continuous representation of $n_B(r, t)$ available from which the v and D profiles can be calculated directly as in [32]. Therefore, smooth v and D profiles have to be found such that the resulting simulated local densities n_S evaluated at the measurement points $n_S(r_l, t_k)$ are in good agreement with the measured data $n_B^{l,k}$. This inverse problem can be cast in the formalism of a minimization. By inserting some initial D and v profiles in equation (1) a forward calculation is performed yielding a simulated density $n_S(r, t)$. The simulated density is evaluated at the discrete measurement points resulting in $n_S(r_l, t_k)$ such that the difference between the measured and simulated density at the respective points can be expressed by $|n_S(r_l, t_k) - n_B^{l,k}|$. By varying the input D and v profiles a new value of $|n_S(r_l, t_k) - n_B^{l,k}|$ is obtained and this process is iterated until a minimum of the difference is found. In this way, the D and v profiles corresponding to the measured boron density $n_B^{l,k}$ is acquired. This minimization procedure can mathematically be written as:

$$\min_{D,v,S} \frac{1}{2} \sum_l \sum_k \left| \frac{n_S(r_l, t_k) - n_B^{l,k}}{\sigma_B^{l,k}} \right|^2 \quad \text{w.r.t equation (1),} \quad (4)$$

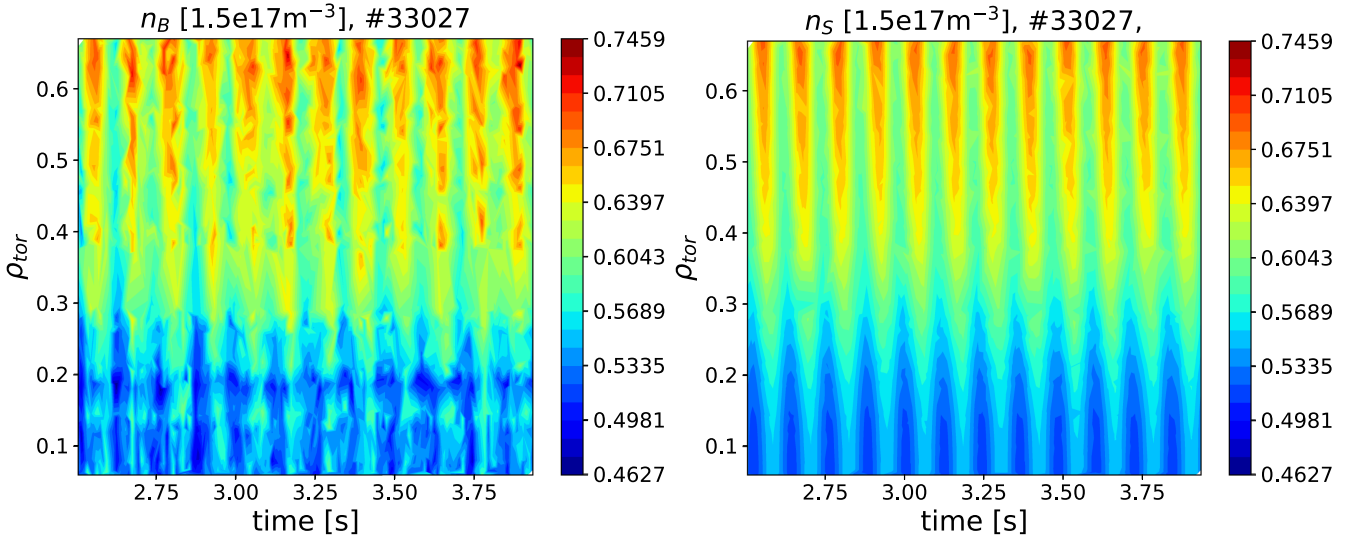


Figure 3. Left: contour plot of the measured boron density. Right: contour plot of the reconstructed boron density. Note that the two plots have the same colorbar.

where $\sigma_B^{l,k} = \sigma_B(r_l, t_k)$ are the standard deviations of the densities at the measurement points r_l and t_k . Since the measured boron signal is periodic (see figure 1), a robust ansatz for the reconstructed density $n_S(r, t)$ as well as the boundary condition $s(t)$ which fits the measured data particularly well is

$$\begin{aligned} n_S(r, t) &= n_0(r) + a(r)\cos(\omega t) + b(r)\sin(\omega t) \\ s(t) &= s_0 + a_0\cos(\omega t) + b_0\sin(\omega t). \end{aligned} \quad (5)$$

In equation (5), n_0 is the steady-state density and $\omega = 2\pi f$, where f is the frequency of the modulation. The boron source term $s(t)$ is assumed to have the same time dependence as the density n_S . This ansatz is used when solving the inverse problem (4) and inserting the ansatz in equation (4) yields

$$\min_{D, v, s_0, a_0, b_0} \frac{1}{2} \sum_l \sum_k \left| \frac{(n_0(r_l) + a(r_l)\cos(\omega t_k) + b(r_l)\sin(\omega t_k)) - n_B^{l,k}}{\sigma_B^{l,k}} \right|^2. \quad (6)$$

By also inserting the ansatz in the radial transport equation (1), three equations are obtained: one for the steady-state $n_0(r)$

$$\frac{1}{r} \frac{\partial}{\partial r} [r(D(r)n_0'(r) - v(r)n_0(r))] = 0 \quad (7)$$

and two for the modulation part $a(r)\cos(\omega t) + b(r)\sin(\omega t)$

$$\omega a(r)r + \frac{\partial}{\partial r} [r(D(r)b'(r) - v(r)b(r))]r = 0, \quad (8)$$

$$-\omega b(r)r + \frac{\partial}{\partial r} [r(D(r)a'(r) - v(r)a(r))] = 0. \quad (9)$$

Inserting the boundary condition $s(r)$ in the transport equations yields

$$n'(0) = 0, \quad n(r_{\max}) = s_0, \quad (10)$$

$$a'(0) = 0, \quad a(r_{\max}) = a_0, \quad (11)$$

$$b'(0) = 0, \quad b(r_{\max}) = b_0, \quad (12)$$

where r_{\max} is the last data point of the experimental data. Finally, the Neumann boundary conditions for D and v , which resolve the singularity at $r = 0$, can be expressed as follows:

$$v(0) = 0, \quad v'(0) = 0, \quad v'(r_{\max}) = 0, \quad (13)$$

$$D'(0) = 0, \quad D'(r_{\max}) = 0, \quad D \geq 0. \quad (14)$$

Equations (6)–(14) represent the complete mathematical description of the minimization problem. The model can thus be reduced to the steady-state and the modulation at the frequency ω , which corresponds to a Fourier transform in time at the modulation frequency. Thus, no time discretization is necessary.

In this work, equation (4) is solved with sequential least squares programming [33], which is a quasi-Newton method, yielding the reconstructed density n_S by solving equation (1) for various D and v profiles as well as the source term s , which is composed of the coefficients s_0 , a_0 , and b_0 . The implementation of the problem has been set up in Python using the Scipy minimization library [34]. Second order finite differences are used for solving the transport equation. The D and v profiles are represented with arbitrary order B-splines, which enforce smoothness of the solution and is therefore a way of applying a regularization to the ill-posed problem. The minimization is thus performed over the B-spline knots of D and v as well as the coefficients s_0 , a_0 , and b_0 . An example of the measured (left) and reconstructed (right) boron density is presented in figure 3. This shows very good agreement. This becomes even clearer when studying the left-hand side of figure 4, which displays the difference between the simulated and measured boron density $|n_S - n_B|$. No additional modulation at the modulation frequency of 8.33 Hz or any other frequencies can be seen and hence, the ansatz from equation (5) is, for this problem, very well suited. This claim is further confirmed by computing a Fourier decomposition of the measured boron density n_B at $\rho_{\text{tor}} = 0.21$ (blue curve on the right-hand side of figure 4), which only displays a peak at

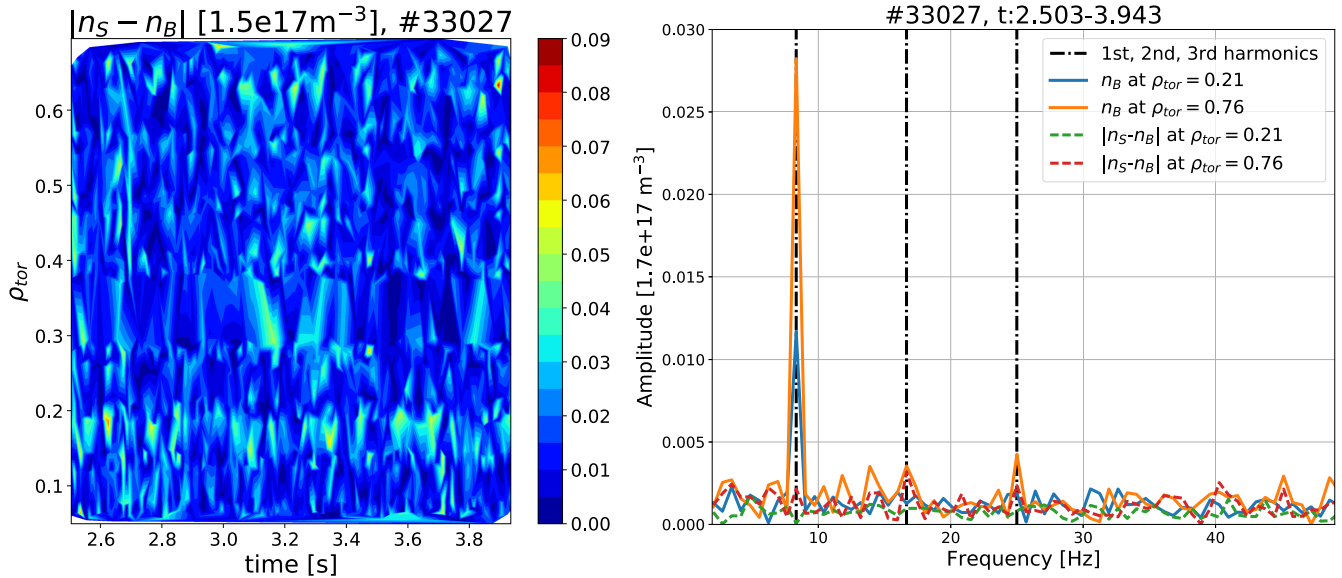


Figure 4. Left: contour plot of the difference between the simulated and measured boron density $|n_S - n_B|$. Right: Fourier decomposition of the measured boron density n_B at $\rho_{\text{tor}} = 0.21$ in solid blue and at $\rho_{\text{tor}} = 0.76$ in solid orange as well as the difference $|n_S - n_B|$ at $\rho_{\text{tor}} = 0.21$ in dashed green and at $\rho_{\text{tor}} = 0.76$ in dashed red. Lines indicating the first, second, and third harmonics in dash-dotted black.

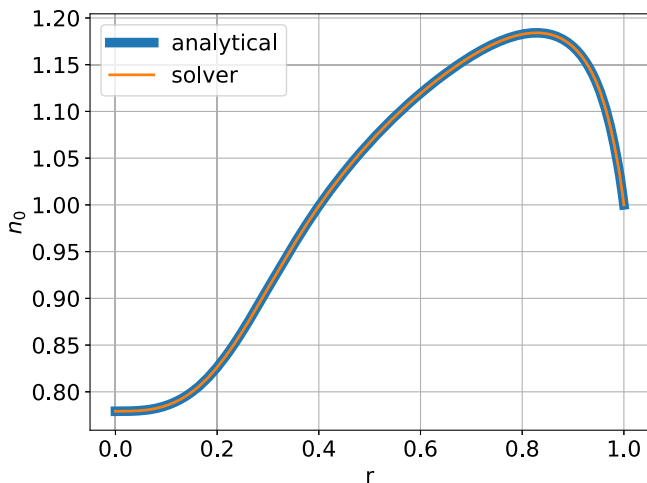


Figure 5. Code verification with the method of manufactured solutions. The blue curve represents the normalized analytically calculated steady-state n_0 and the orange curve is the steady-state acquired from our radial transport solver.

the modulation frequency at 8.33 Hz, hence no higher harmonics are present in the central region of the plasma. In orange the same quantity at $\rho_{\text{tor}} = 0.76$ is shown. At this location near the plasma edge, additional smaller peaks are visible at the second and third harmonics, indicated by the black dash-dotted lines. These peaks are, however, one order of magnitude smaller than the peak at the first harmonics, i.e. 8.33 Hz, and barely distinguishable from the noise. Furthermore, the Fourier decompositions of the differences between the simulated and measured boron density $|n_S - n_B|$ at $\rho_{\text{tor}} = 0.21$ and $\rho_{\text{tor}} = 0.76$ are plotted with dashed green and red lines, respectively, and this quantity shows no additional frequency peaks outside of the noise.

In the dataset presented here, sawteeth are present with a frequency of 16 Hz and an inversion radius of $\rho_{\text{tor}} \sim 0.25$. That corresponds to ~ 1.6 sawtooth cycle for every CXRS integration time. The experimental data inside of $\rho_{\text{tor}} < 0.25$ is, hence, a sawtooth average, and therefore, the sawtooth frequency cannot be seen in the right plot of figure 4.

3. Method validation and uncertainties

3.1. Method validation

With the new ICRF modulation technique multiple modulation cycles can be measured and analyzed together in an otherwise constant background plasma, see figure 3. This is a big advantage compared to other techniques such as laser blow-off and gas puffing, which commonly determine the transport coefficients by analyzing only one single cycle, for example, one individual laser blow-off. The method presented here reduces the relative noise and has a lower statistical uncertainty.

Before using the numerical simulation tool to predict the transport coefficients, it is important to build trust in its reliability and make sure the solver has been implemented correctly. This can be done by checking whether the simulation tool accurately reproduces an analytical solution. This approach is called the method of manufactured solutions [35, 36]. The procedure is straight-forward: suppose the D and v profiles have been found. The transport equation can then be solved analytically for the steady-state $n_0(r)$ by integrating equation (2) from r to r_{max} :

$$\int_r^{r_{\text{max}}} \frac{v(r')}{D(r')} dr' = \int_r^{r_{\text{max}}} \frac{1}{n_0(r')} \frac{\partial n_0(r')}{\partial r'} dr' = [\log(n_0(r'))]_r^{r_{\text{max}}} = \log(n_0(r_{\text{max}})) - \log(n_0(r)). \quad (15)$$

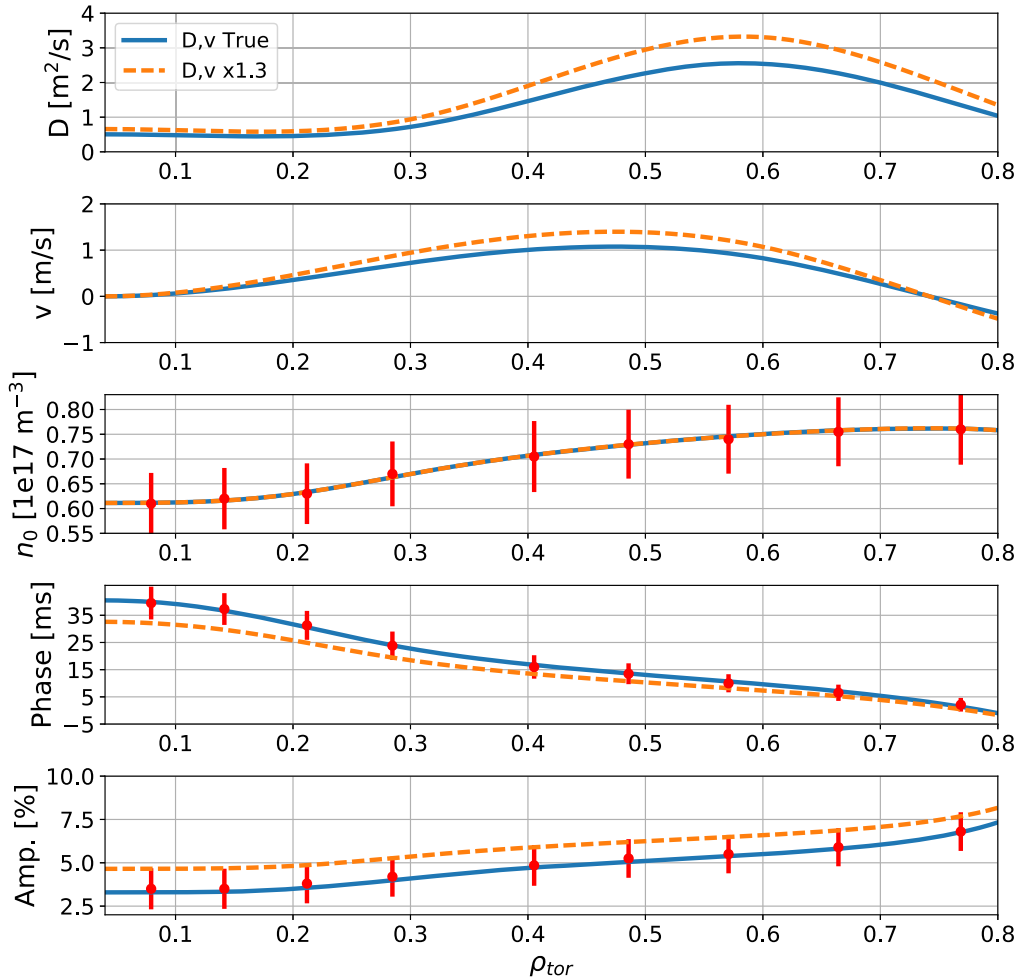


Figure 6. Synthetic D and v profiles and the resulting steady-state, phase, and amplitude profiles. Dashed orange lines: Blue D and v (called D , v True) profiles multiplied by a factor of 1.3. Example experimental data points and error bars in red to display what kind of features we are able to measure with our CXRS systems. A change resulting from a smaller factor than 1.3 in the transport coefficients cannot be distinguished outside the error bars with the CXRS diagnostic.

Expanding both sides yields:

$$\Rightarrow \exp\left(-\int_r^{r_{\max}} \frac{v(r')}{D(r')} dr'\right) = \frac{n_0(r)}{n_0(r_{\max})} = \frac{n_0(r)}{s_0}, \quad (16)$$

$$n_0(r) = \exp\left(-\int_r^{r_{\max}} \frac{v(r')}{D(r')} dr'\right) s_0. \quad (17)$$

By inserting analytical D and v profiles in equation (17), an analytical expression for $n_0(r)$ can be calculated. The analytical $n_0(r)$ is compared to the $n_0(r)$ obtained by inserting the same analytical D and v profiles into the second order finite difference solver for the transport equation. If the two different approaches give the same $n_0(r)$, the radial transport solver has been correctly implemented. This test was performed and the normalized analytical steady-state $n_0(r)$ (blue) and the steady-state $n_0(r)$ obtained from the solver (orange) are presented in figure 5. The agreement is perfect and it can, thus, be concluded that the radial transport solver has been correctly implemented.

To investigate what features can be resolved in the transport coefficient profiles with the AUG CXRS diagnostics, further method validation was performed. Synthetic D and v profiles

were given to the radial transport solver and the steady-state n_0 density, phase and amplitude profiles were calculated (see section 4 on how the phase and amplitude are calculated). In the following examples, the frequency of the modulation was 10 Hz.

First, the D and v profiles (blue lines called True in figure 6) were scaled with a factor of 1.3 (dashed orange lines in figure 6). Additionally in red, example experimental data points with typical error bars are plotted for the steady-state density, phase, and amplitude to display what kind of features we are able to measure with our CXRS systems. It should be noted that our CXRS diagnostic has three times as many data points, but for the sake of clarity only a few are shown. As already discussed in the introduction, scaling both the D and v profiles with the same factor does not influence the steady-state density profile, since it depends on the ratio v/D , but it does affect the phase and amplitude profiles. When increasing the transport coefficients, the phase shift decreases meaning the modulation propagates more quickly from the edge into the core. In this example, multiplying the transport coefficients by a factor of 1.3 results in a change of the phase shift from ~ 40 to ~ 30 ms. Additionally, the amplitude of the

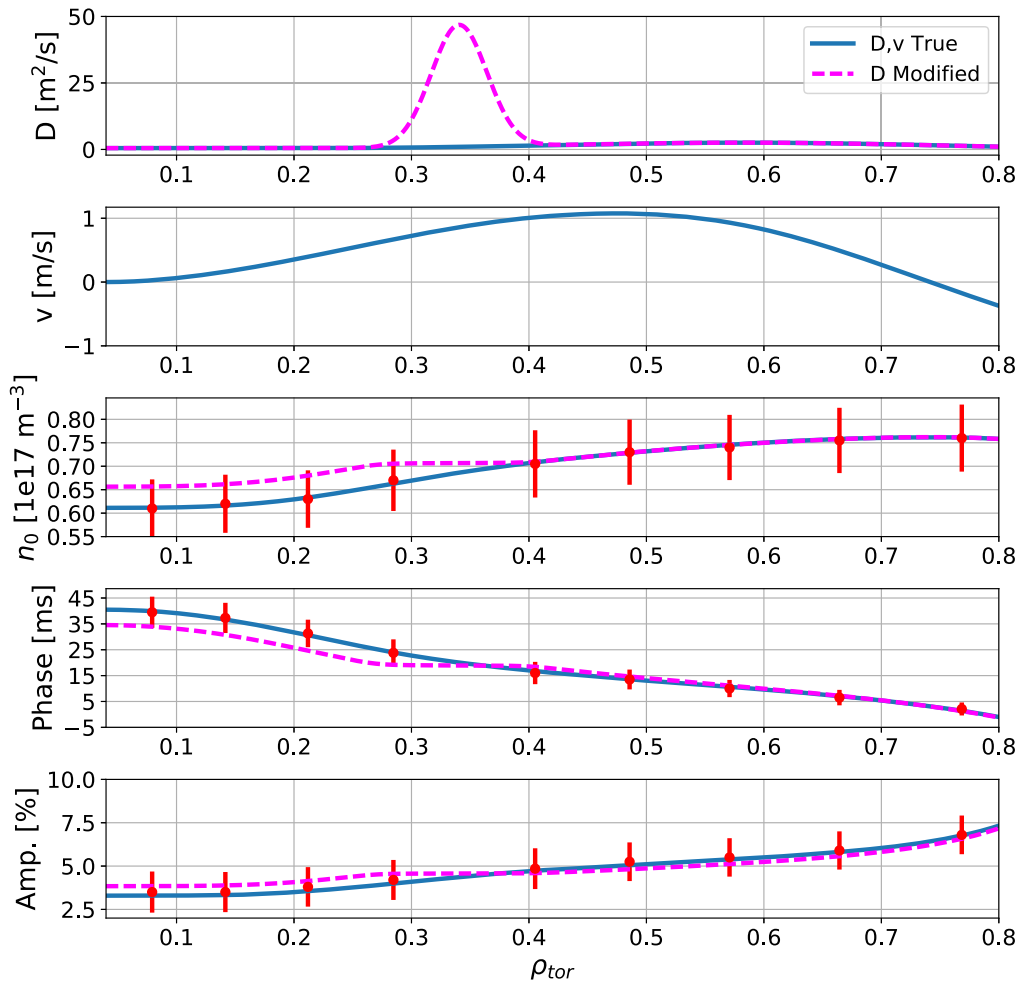


Figure 7. Synthetic D and v profiles and the resulting steady-state, phase, and amplitude profiles. To simulate a MHD mode, a peak in the D profile has been added (dashed magenta lines) on top of the original profile (blue lines). The v profile was kept constant. Example experimental data points and error bars in red to display what kind of features we are able to measure with our CXRS systems.

modulation is less damped compared to the case with the smaller D and v profiles. Hence, scaling the transport coefficients with a larger factor would result in a faster inward propagation and less damping. Judging from the error bars on the phase and amplitude in figure 6, a change resulting from a smaller factor than 1.3 in the transport coefficients cannot be distinguished outside of the error bars with the CXRS diagnostic.

The second task was to investigate how a potential MHD mode could affect the profiles. Such a mode could, for example, enhance the transport locally and, thus, give rise to a sharp localized peak in the D profile. The resultant steady-state density, phase and amplitude profiles from such hypothetical D profiles (keeping the v profile constant) are shown in figure 7. The blue lines corresponds to the same profiles shown in figure 6. For the dashed magenta lines, a sharp Gaussian peak in D has been added around $\rho_{\text{tor}} \sim 0.33$, to simulate a 3/2 neoclassical tearing mode [37]. This sharp rise in the D profile leads to a flattening of the steady-state profile n_0 at $\rho_{\text{tor}} \sim 0.33$ as well as in the phase and the amplitude profiles. This flattening is still within the experimental error bars for the steady-state and the amplitude, but on the

borderline for the phase. It is also interesting to investigate how the sharpness of such a peak affects the profiles. The result of such an investigation is displayed in figure 8. The blue lines are, again, the original profiles and the width of the added Gaussian peak has consequently been increased for the dashed green, magenta, and orange lines. One can see that increasing the width also leads to a flattening of the steady-state, phase, and amplitude profiles. Here, the green case is well within the experimental error bars, the magenta case is borderline for the steady-state and phase, but the orange case is outside the error bars for the steady-state and phase. It is obvious to see that such a small feature in the dashed green steady-state profile can probably not be distinguished from the blue case within the error bars, whereas it should not be a problem to distinguish between the orange and the blue cases. However, given the width of the peak in the magenta and orange case, the peak cannot be classified as localized anymore. The fact that a very large change in the D or v profile only leads to small changes in the experimental data, that is in the steady-state, phase, and amplitude, stems from the ill-posedness of the inverse problem. This further strengthens the argument that when solving such an inverse problems, some

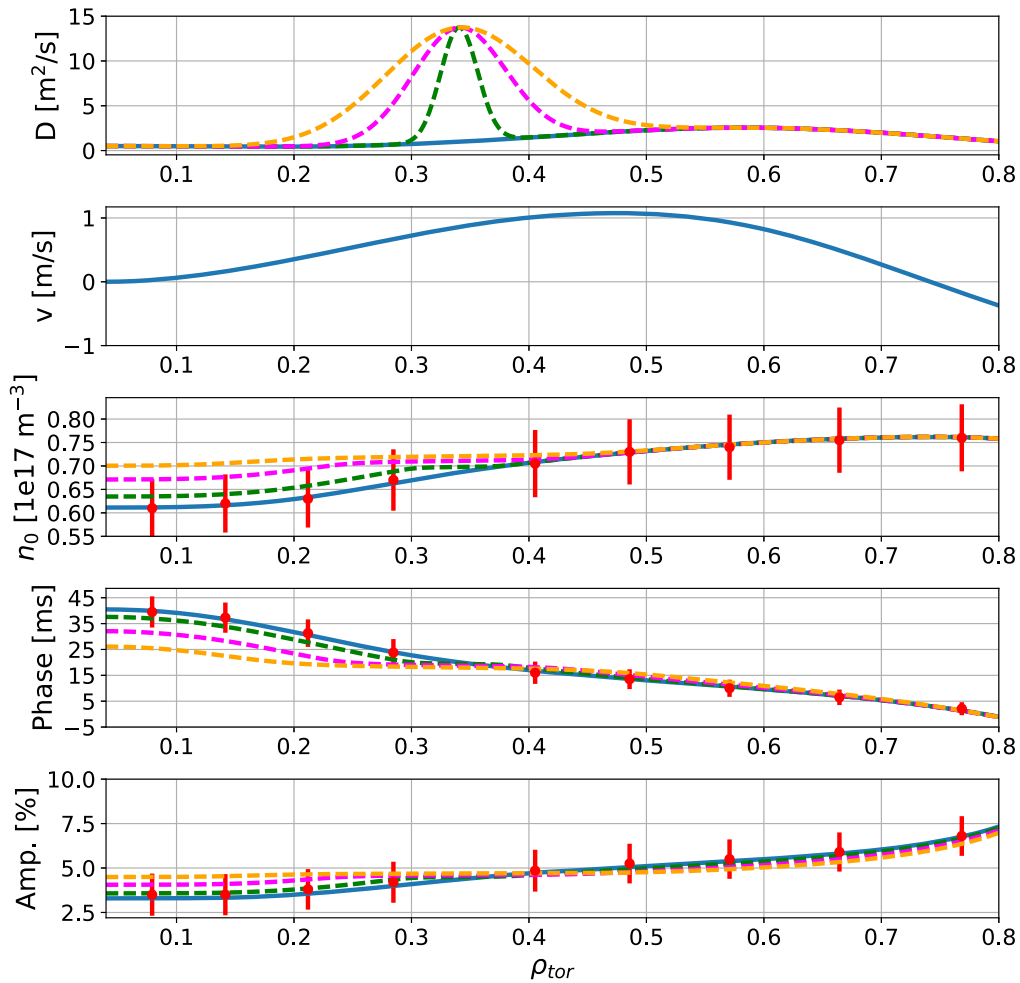


Figure 8. Synthetic D and v profiles and the resulting steady-state, phase, and amplitude profiles. To simulate a MHD mode, peaks in the D profile with different widths have been added (dashed green, magenta, and orange lines) on top of the original profile (blue case). The v profile was kept constant. Example experimental data points and error bars in red to display what kind of features we are able to measure with our CXRS systems.

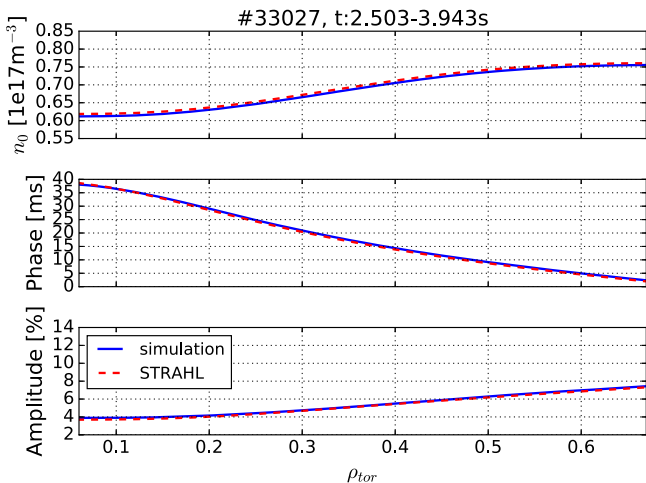


Figure 9. Result of the benchmark of our radial transport solver (blue) with the impurity transport code STRAHL (red). There is a very good agreement in all quantities between the two codes.

kind of regularization must be applied in order to heal the ill-posedness. To conclude, it would be difficult to distinguish a mode outside of the error bars with our CXRS systems, since such a mode has to cause a peak with a very high diffusion

and/or be very broad, in which case the mode would not be localized anymore.

As an additional check, our radial transport solver was benchmarked against the impurity transport code STRAHL [12], which solves the forward problem. The final D and v profiles from our solver were given to STRAHL as inputs to cross-check the simulated steady-state density n_0 and the phase and amplitude of the modulation (see section 4 on how the phase and amplitude are calculated). As can be seen in figure 9, there is an excellent agreement in all quantities between the two codes. However, the radial transport solver developed in this work was able to perform one forward calculation in a few milliseconds, whereas STRAHL, which deploys an additional computational expensive time discretization using the Crank–Nicolson scheme, needs several seconds for the same task. In fact, in some cases the complete minimization procedure is finished in the same time it takes STRAHL to complete one forward calculation. The fact that the inverse problem can be solved in few seconds makes it possible to carry out a brute force uncertainty estimation, described in the next section, in a couple of hours. For such analysis STRAHL would require several days.

3.2. Uncertainties

As mentioned in section 2.1, the calculation of the experimental boron density via equation (3) is subjected to several different uncertainties. The first one is the error on the measured CXRS line intensity, which is well known and characterized. The second one is the uncertainty on the atomic data that goes into the effective charge exchange emission rates $\langle\sigma v\rangle$, which is estimated at $\sim 15\%$ – 20% . The changes in $\langle\sigma v\rangle$ are dominated by the interaction energy, which is constant in our case. Changes due to the electron density, ion, and electron temperatures along the beams are minimal, and, therefore, this uncertainty does not significantly impact the error on the overall profile shape, which is of interest in our case. The third and final one is the uncertainty in the determination of the neutral beam density. This in turn depends on the uncertainties of the beam stopping rates, the beam geometry as well as the measured electron density. The same argument as above applies to the beam stopping rates, i.e. they do not significantly affect the profile shape error bar and have, therefore, been neglected. The beam geometry is well diagnosed and benchmarked by the two different NBI systems as well as by cross-checking that the same answer is obtained when using different beam combinations. The electron density n_e has the biggest impact on the attenuation of the neutral beam. If the standard deviation on the electron density was known, this uncertainty could be added to the overall uncertainty on the neutral beam density. However, the electron density is deduced from an integrated data analysis framework [38], where the measured electron densities from several different diagnostics have been combined into one profile using Bayesian inference and this procedure does not produce a standard deviation, but rather confidence bands on the electron density, which cannot be used in a standard error propagation scheme. Therefore, this last source of error is not routinely included in the overall uncertainty on the impurity density, but can be tested by running the analysis with variations of the electron density.

The measurement data $n_B(r_l, t_k) = n_B^{l,k}$ is, therefore, subjected to an error such that each data point is normally distributed $\mathcal{N}(n_B^{l,k}, \sigma_B^{l,k})$ with standard deviation $\sigma_B^{l,k}$. The relative error on the boron density is difficult to assess (see discussion above) and, therefore, the error on the boron intensity $I_B^{l,k}$ is used for $\sigma_B^{l,k}$. To assess the uncertainties on the measured transport coefficients, a full Monte Carlo approach is used, where this multi variate normal distribution of the experimental data $\mathcal{N}(n_B^{l,k}, \sigma_B^{l,k})$ is assumed. $N_S = 10\,000$ random samples are drawn from this distribution and every sample goes through the simulation procedure described in section 2.3, thus resulting in a posterior distribution. The posterior distribution itself, however, is not normally distributed even though the samples were drawn from a normal distribution. The reason for this is because the transport equation is nonlinear. The uncertainty bands of the D and ν profiles shown in the next section are pointwise confidence intervals of 95% of the posterior distribution. These confidence intervals of the D and ν profiles, therefore, only represent a statistical error. Other non-statistical uncertainties

most certainly also play a role, but require a full Bayesian framework, which is outside the scope of this work.

4. Experimental results and comparison to theory

The experimental data presented here is from an H-mode plasma with a toroidal magnetic field of 2.5 T at the magnetic axis, a plasma current of 0.8 MA, and an edge safety factor q_{95} of 5.2. The heating scheme was as follows: 5 MW of constant NBI, 0.5 MW of constant ECRH, and 1 MW modulation of the ICRF heating power as described in section 2.1. Time traces of the heating powers can be seen in figure 10(c). The modulation frequency of the ICRF heating power, and consequently the frequency of the boron modulation, was 8.33 Hz, which can be seen in figure 1. As mentioned in section 2.1, for this technique to be valid the modulation of the background plasma should be kept to a minimum. Consequently the amplitude of the modulation of the electron density n_e ($<1\%$), electron temperature T_e ($<1\%$), and ion temperature T_i (3%) at the radial location $\rho_{\text{tor}} \sim 0.1$ were examined. Time traces of these signals as well as of the plasma stored energy (WMHD) are shown in figures 10(a), (b), and (d). Note that these $<1\%$ and 3% temperature modulations are near the plasma center where the ICRF is deposited and where the perturbation is the largest. The phase and amplitude profiles of the T_e (orange) and T_i (blue) modulations are shown in figure 11 and one can see that they are consistent with direct heating by the ICRF as they peak in the center and have a minimum in phase at this location as well. The phase of for T_e is not shown as it is within the noise level and, therefore, not meaningful. We define our region of interest to start at $\rho_{\text{tor}} = 0.25$, since these quantities are $<1\%$ outside of this location. The even smaller perturbation observed in the n_e profile is a very good sign that transport is not changing as a result of the applied perturbations.

The steady-state boron density profile n_0 as well as the phase and amplitude profile of the boron modulation is presented in figure 12. The red points are the mean experimental data points. By fitting the experimental data $n_B(r_l, t_k)$ (left-hand side of figure 3) with the function presented in equation (5), the phase and amplitude profiles can be calculated from the coefficients $a(r)$ and $b(r)$, where the amplitude is given by $\sqrt{a(r)^2 + b(r)^2}$ and the phase by $\arctan(a(r)/b(r))$. This will give a phase in units of radians, but it is more intuitive to have the phase in units of seconds, since it represents the propagation time of the modulation. The conversion from radians to seconds can be obtained by dividing the phase, in radians, with $2\pi f$, where f is the frequency of the modulation. The steady-state profile is simply given by $n_0(r)$ in equation (5). By performing the same exercise on the reconstructed data shown on the right-hand side of figure 3, these quantities can be extracted and are represented by the blue lines in figure 12. Hence, the blue lines are not fits to the red data points, but rather the results of the minimization. The agreement is very good, which is a sign of the correctness of the method used. One can note several

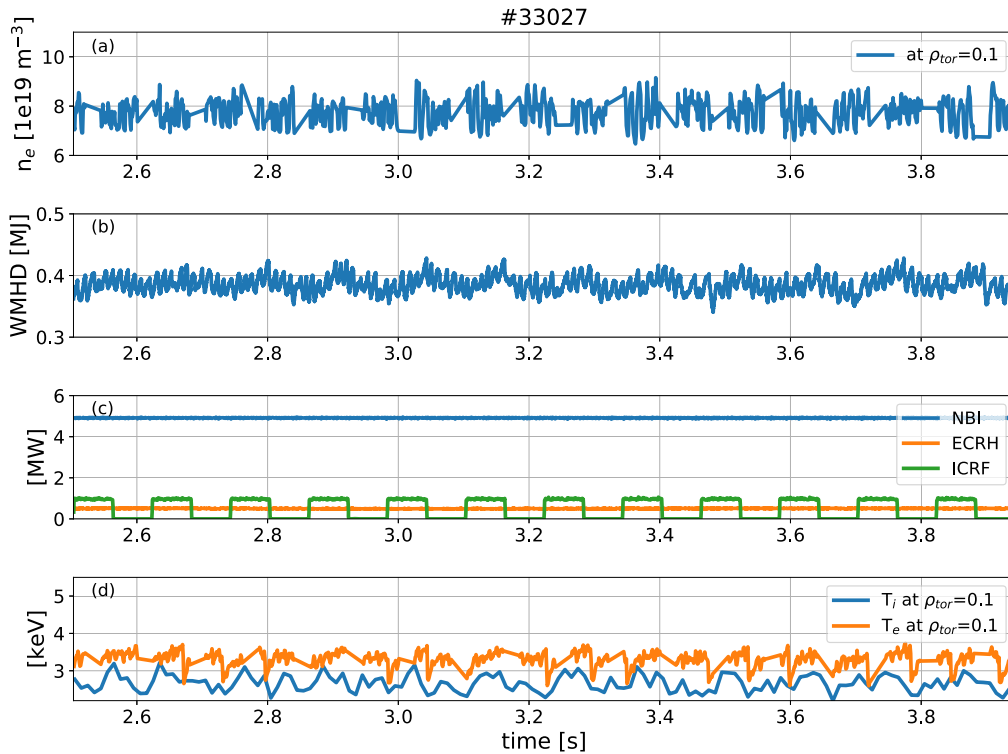


Figure 10. Time traces of the (a) electron density at $\rho_{\text{tor}} = 0.1$, (b) plasma stored energy (WMHD), (c) NBI, ECRH, and ICRF heating powers, and (d) electron and ion temperature at $\rho_{\text{tor}} = 0.1$ for the dataset presented in this paper.

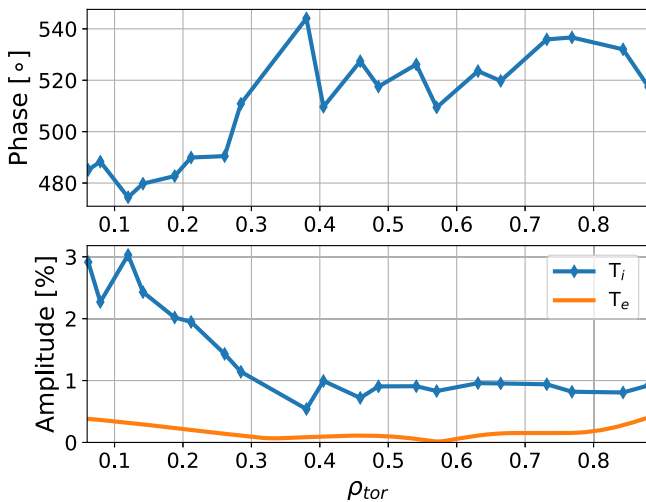


Figure 11. Phase and amplitude of the modulation of T_e (orange) and T_i (blue) at the modulation frequency 8.33 Hz. The phase of for T_e is not shown as it is within the noise level.

things when studying figure 12. First, in this particular case the steady-state boron density profile is hollow. Second, the phase shift indicates how fast the modulation propagates into the core. In this case the phase shift is ~ 35 ms. Third, the amplitude modulation is strongest at the edge where the boron source is located, which also becomes clear by looking at figure 3. In this case, the amplitude of the modulation is $\sim 7\%$ at $\rho_{\text{tor}} \sim 0.7$.

The deduced transport coefficients are compared with neoclassical calculations, performed with the code NEO [39, 40], as well as quasi-linear gyrokinetic simulations at a

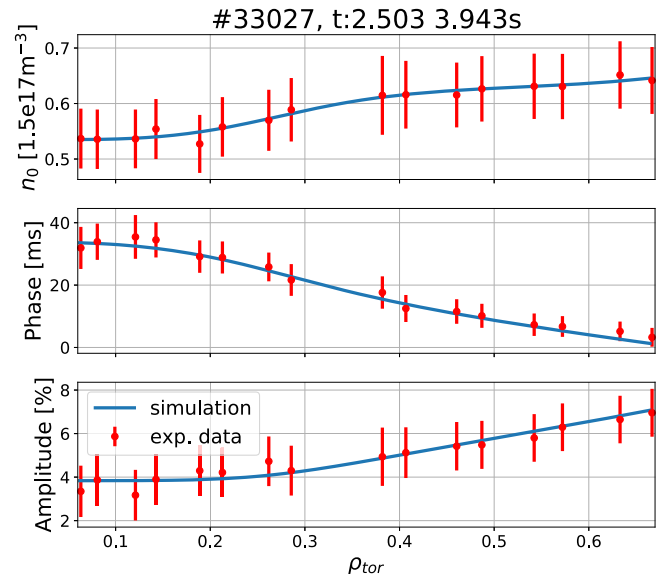


Figure 12. Steady-state, phase, and amplitude of the measured data (red) and the simulation (blue). The blue lines are not fits to the red data points, but rather the results of the minimization.

wavenumber $k_\theta = 0.4$, computed with the code GWK [41]. Turbulent and neoclassical transport components are summed under the assumption that the turbulent transport heat conductivity matches the anomalous part of the power balance heat conductivity, which is computed with TRANSP [42] (see [43] for more details).

The experimental transport coefficients for the dataset presented in this paper as well as a first comparison to theory

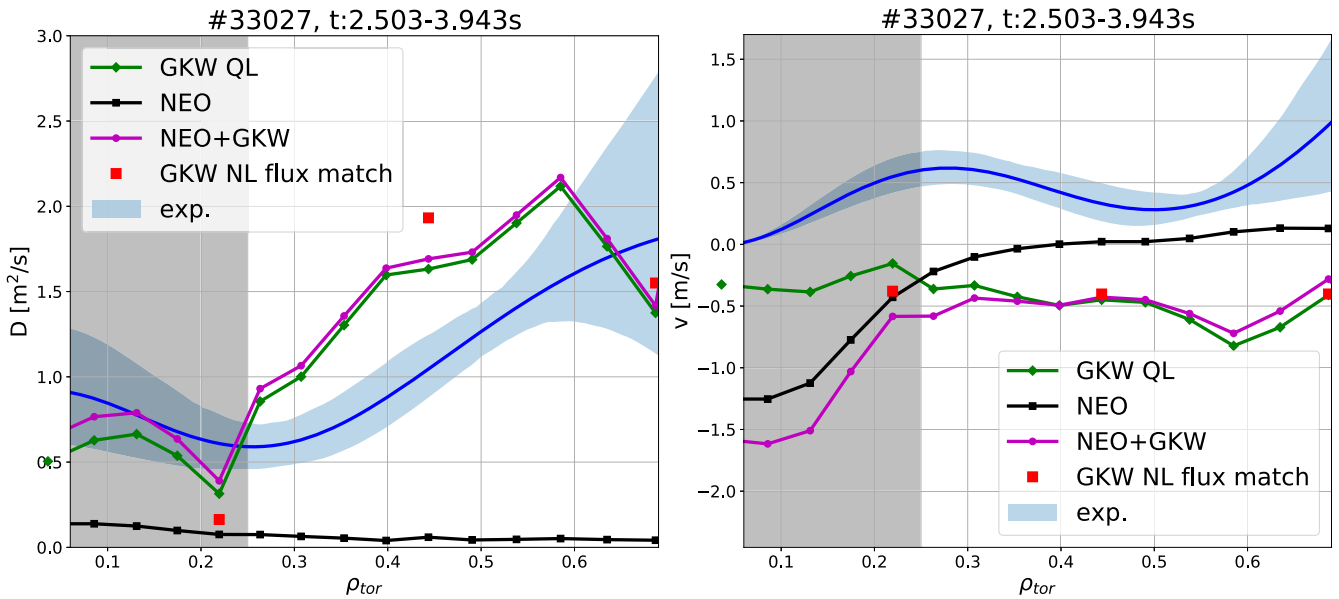


Figure 13. Experimental and theoretical D (left) and v (right) profiles. The experimental profiles with their uncertainty bands are depicted in blue, the ones from the quasi-linear GKW run in green diamonds, from NEO in black squares, and the total theoretical profiles in magenta circles. The red squares represent the result of the nonlinear GKW run performed with matching of the heat fluxes. Our region of interest starts at $\rho_{\text{tor}} = 0.25$, hence, where the gray region ends.

are presented in figure 13. Neoclassical diffusion (black squares) is, as expected, much smaller than the measured values (blue curve), and hence, the transport is turbulence driven. In this case, the total theoretical D (magenta points) agrees reasonably well within a factor of 2 with the experimental one, whereas the theoretical v is in the opposite direction. The sign convention of v is the following: a hollow density profile means an outward drift velocity e.g. $v > 0$. The opposite ($v < 0$) corresponds to an inward directed drift velocity, which means a peaked density profile. The experimental v profile is positive (outward), which agrees with the hollow steady-state n_0 profile in figure 12. Since the theoretical drift velocity is negative, the theory fails to capture this and instead predicts considerably more peaked steady-state profiles than are measured. The fact that the gyrokinetic simulation is unable to fully capture the hollowness of the steady-state profile is consistent with previous experimental studies on carbon transport [44–46] and nitrogen transport [46] at JET and steady-state boron transport at AUG [14, 43, 47], which also observed discrepancies between theory and experiment for hollow impurity density profiles. The uncertainty regions of the D and v profiles in figure 13 are calculated as described in section 3. The end of the gray region indicates where our region of interest starts ($\rho_{\text{tor}} = 0.25$).

To investigate how large an impact the ion temperature modulation has on the transport coefficients calculated by GKW, additional simulations with an averaged maximum and an averaged minimum T_i profile were performed. Maxima and minima of the T_i modulation were chosen and from these profiles an average maximum and minimum T_i profile were created, which were then used as inputs to GKW. The resultant transport coefficients of the new runs (red diamonds and yellow squares) closely follow the original run (blue) except at $\rho_{\text{tor}} \sim 0.22$, which is represented by the error bar in

figure 14. The reason for the different results is that the point at this location is highly sensitive to the choice of the spectrum in the calculation of the quasi-linear transport. Different branches are competitively almost equally unstable, but produce very different impurity transport. It can thus be concluded that, overall, the change in the ion temperature gradient due to the small modulation present does not impact the predicted transport coefficients from GKW. As a consistency check, nonlinear GKW runs, both with and without heat flux matching between theory and experiment, at three radial locations were performed. The results of these runs can be seen in figure 15, where D/χ_i (top) and v/χ_i (bottom) profiles are plotted. The quasi-linear results are shown in green, the nonlinear without heat flux matching in magenta, and the nonlinear with heat flux matching in red. As can be seen, the nonlinear results agree well with the quasi-linear results, except for the data point at $\rho_{\text{tor}} \sim 0.22$, but this comes as no surprise since that point is much more sensitive to the choice of the k_θ spectrum (see figure 14 and discussion above) and the nonlinear result includes all unstable modes, while the quasi-linear one is only based on the most unstable [16]. Additionally in figure 15, the experimental profiles are shown in blue. For D/χ_i , the experimental result is on the order of unity. While in the dimensionless parameters D/χ_i and v/χ_i there is a good correspondence between nonlinear and quasi-linear results in figure 15, some differences are particularly visible when the comparison is performed on the dimensional nonlinear with flux matching (red squares) and the reconstructed quasi-linear values (green points) of D in figure 13, as the ion temperature gradients which have been used to match the heat flux had to be modified with respect to the nominal values. This is particularly visible for the D values around mid-radius, for which the logarithmic ion temperature gradient had to be decreased by almost 20%, with a

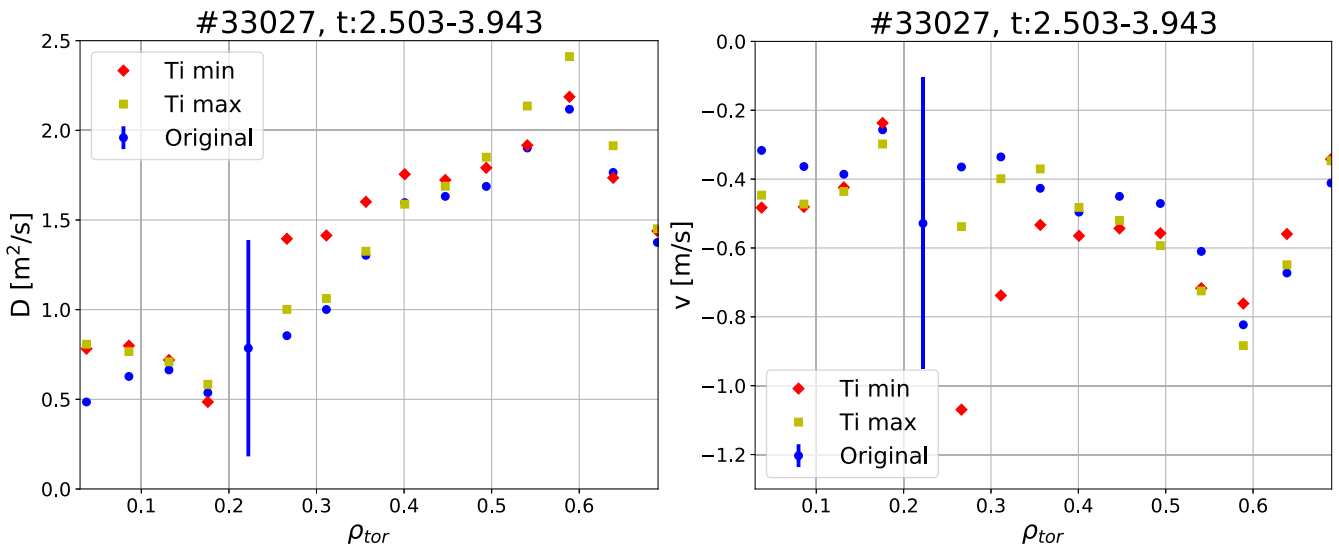


Figure 14. D (left) and v (right) profiles (blue points) from GKW. The error bar at $\rho_{\text{tor}} \sim 0.22$ shows how D and v change in the runs with the average maximum and minimum T_i profile at this particular location. The rest of the data points from these additional runs are shown as red diamonds (T_i minimum) and yellow squares (T_i maximum) and show significantly less variation than at $\rho_{\text{tor}} \sim 0.22$.

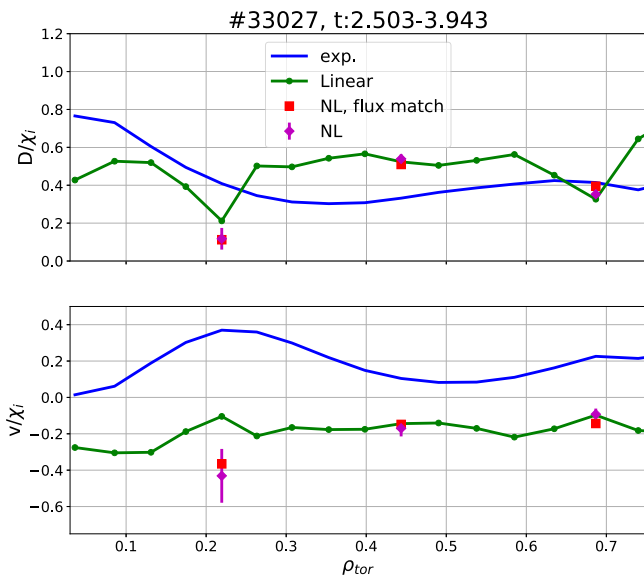


Figure 15. D/χ_i (top) and v/χ_i (bottom) profiles from GKW comparing the results from the quasi-linear (green line) with the nonlinear without flux matching (magenta diamonds), and nonlinear with flux matching (red squares) runs. The experimental profiles are shown in blue.

consequent increase of the local value of the corresponding heat conductivity. In contrast, for the inner and outer points, the logarithmic ion temperature gradients had to be increased by less than 10% in order to match the anomalous part of the ion heat flux computed with TRANSP.

5. Conclusion

A new ICRF modulation technique has been developed and exploited at AUG for obtaining time-dependent boron density signals. The technique has been thoroughly tested and applied





in several different plasma conditions, one example is shown in this paper. This method requires CXRS data of good quality, i.e. high spatial and temporal resolution. From the CXRS measured boron intensity, the boron density is calculated using effective charge exchange emission rate coefficients and neutral beam densities. With a time-dependent boron density signal at hand, the boron transport coefficients D and v in the radial transport equation are individually determined. This task is done by solving an inverse problem by a quasi-Newton method. The functional form assumed for the boron density is a background steady-state component plus a sum of a cosine and sinus with the frequency of the modulation. Hence, a second requirement is that the boron density signal can be well represented by the sum of a few sinusoidal terms. This implemented framework has been verified with the method of manufactured solutions as well as benchmarked with the transport code STRAHL. The uncertainties of the transport coefficient profiles are estimated with a Monte Carlo procedure, where random samples are drawn from a multi variate normal distribution of the experimental data. The transport coefficients are assumed to be constant in time, which requires a constant plasma background. This implies that the resultant modulation of the electron density, electron temperature, and the ion temperature should be kept as small as possible. It was observed that the modulation of the electron density, temperature, and the ion temperature are less than 1% in the radial region of interest ($\rho_{\text{tor}} > 0.25$), whereas the boron density modulation amplitude is 4% in the core and 7% at the edge. The experimental transport coefficients are compared with neoclassical calculations with the code NEO and quasi-linear and nonlinear gyrokinetic simulations with the code GKW. Neoclassical D is well below the experimental D meaning the transport is turbulence driven. The comparison to gyrokinetic theory shows an agreement within a factor of 2 in D , but the theoretical v has the opposite sign, i.e. predicts a more peaked steady-state boron density

profile than measured in the experiment. Additional nonlinear GKW runs have been carried out and the nonlinear results agree well with the linear. The analysis, thus, enables us to measure core boron transport coefficients over a wide range of plasmas with error bars on the order of $\sim 30\%$. Moreover, this technique and solver are directly applicable in other situations as long as the studied impurity can be measured with high spatial and temporal resolution and there exists a way of modulating the impurity density at the edge in such a way that the resultant density signal is of a sinusoidal-like form. Future work will exploit this method and focus on building up a database of transport coefficients in different plasma conditions, to perform a more comprehensive comparison with the theoretical predictions, and to better characterize the domains of agreement and disagreement.

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References

- [1] Howard N *et al* 2012 Measurement of plasma current dependent changes in impurity transport and comparison with nonlinear gyrokinetic simulation *Phys. Plasmas* **19** 056110
- [2] Grierson B A *et al* 2015 Impurity confinement and transport in high confinement regimes without edge localized modes on DIII-D *Phys. Plasmas* **22** 055901
- [3] Giroud C *et al* 2007 Method for experimental determination of Z dependence of impurity transport on JET *Nucl. Fusion* **47** 313
- [4] Sertoli M *et al* 2011 Local effects of ECRH on argon transport in L-mode discharges at ASDEX Upgrade *Plasma Phys. Control. Fusion* **53** 035024
- [5] Wade M *et al* 1995 Helium transport and exhaust studies in enhanced confinement regimes in DIII-D *Phys. Plasmas* **2** 2357–65
- [6] Dux R *et al* 2003 Influence of the heating profile on impurity transport in ASDEX Upgrade *Plasma Phys. Control. Fusion* **45** 1815
- [7] Rice J *et al* 2015 Core impurity transport in Alcator C-Mod L-, I- and H-mode plasmas *Nucl. Fusion* **55** 033014
- [8] Puiatti M *et al* 2006 Analysis of metallic impurity density profiles in low collisionality Joint European Torus H-mode and L-mode plasmas *Phys. Plasmas* **13** 042501
- [9] Valisa M *et al* 2011 Metal impurity transport control in JET H-mode plasmas with central ion cyclotron radiofrequency power injection *Nucl. Fusion* **51** 033002
- [10] Dux R 2004 Impurity transport in tokamak plasmas Habilitation Universität Augsburg (<http://hdl.handle.net/11858/00-001M-0000-0027-2036-E>)
- [11] Hinton F and Hazeltine R 1976 Theory of plasma transport in toroidal confinement systems *Rev. Mod. Phys.* **48** 239
- [12] Dux R 2014 Strahl User Manual Report IPP 10/30 (<http://hdl.handle.net/11858/00-001M-0000-0027-0DB8-4>)
- [13] Pereverzev G V and Yushmanov P 1991 ASTRA. Automated System for TRansport Analysis in a tokamak IPP 5/42 (<http://hdl.handle.net/11858/00-001M-0000-0027-646C-F>)
- [14] Kappatou A 2014 Investigations of helium transport in ASDEX Upgrade plasmas with charge exchange recombination spectroscopy *PhD Thesis Technische Universiteit Eindhoven* (<http://hdl.handle.net/11858/00-001M-0000-0029-5701-F>)
- [15] Angioni C *et al* 2014 Tungsten transport in JET H-mode plasmas in hybrid scenario, experimental observations and modelling *Nucl. Fusion* **54** 083028
- [16] Angioni C *et al* 2016 Gyrokinetic study of turbulent convection of heavy impurities in tokamak plasmas at comparable ion and electron heat fluxes *Nucl. Fusion* **57** 022009
- [17] Herrmann A and Gruber O 2003 Chapter 1: ASDEX Upgrade-Introduction and overview *Fusion Sci. Technol.* **44** 569–77
- [18] Streibl B *et al* 2003 Chapter 2: machine design, fueling, and heating in ASDEX Upgrade *Fusion Sci. Technol.* **44** 578–92
- [19] Perkins F 1989 Radiofrequency sheaths and impurity generation by ICRF antennas *Nucl. Fusion* **29** 583
- [20] Manning H *et al* 1986 Impurity generation during ICRF heating experiments on Alcator C *Nucl. Fusion* **26** 1665
- [21] Myra J, D'Ippolito D and Gerber M 1990 Faraday screen sheaths and impurity production during ion cyclotron heating *Nucl. Fusion* **30** 845
- [22] D'Ippolito D *et al* 1991 A model of sheath-driven impurity production by ICRF antennas *Plasma Phys. Control. Fusion* **33** 607
- [23] Janzer A 2015 Tungsten transport in the plasma edge at ASDEX upgrade *PhD Thesis Ludwig-Maximilians-Universität München* (<http://nbn-resolving.de/urn:nbn:de:bvb:19-187453>)
- [24] Bobkov V *et al* 2016 First results with 3-strap ICRF antennas in ASDEX Upgrade *Nucl. Fusion* **56** 084001
- [25] Winter J *et al* 1989 Boronization in TEXTOR *J. Nucl. Mater.* **162** 713–23
- [26] Rohde V *et al* 1999 Comparison of boronization and siliconization in ASDEX Upgrade *26th EPS Conf. on Controlled Fusion and Plasma Physics* (European Physical Society)
- [27] Viezzer E *et al* 2012 High-resolution charge exchange measurements at ASDEX Upgrade *Rev. Sci. Instrum.* **83** 103501
- [28] McDermott R *et al* 2017 Extensions to the charge exchange recombination spectroscopy diagnostic suite at ASDEX Upgrade *Rev. Sci. Instrum.* **88** 073508
- [29] Fonck R, Darrow D and Jaehnig K 1984 Determination of plasma-ion velocity distribution via charge-exchange recombination spectroscopy *Phys. Rev. A* **29** 3288
- [30] Isler R 1994 An overview of charge-exchange spectroscopy as a plasma diagnostic *Plasma Phys. Control. Fusion* **36** 171

- [31] Summers H P 2004 *The ADAS User Manual, version 2.6* <http://adas.ac.uk>
- [32] Takenaga H *et al* 1998 Determination of particle transport coefficients in reversed shear plasma of JT-60U *Plasma Phys. Control. Fusion* **40** 183
- [33] Kraft D 1988 A software package for sequential quadratic programming *Forschungsbericht- Deutsche Forschungs- und Versuchsanstalt für Luft- und Raumfahrt*
- [34] Jones E *et al* 2001 SciPy: open source scientific tools for Python <http://scipy.org/>
- [35] Salari K and Knupp P 2000 Code verification by the method of manufactured solutions *Technical Report Sandia National Laboratories* (<https://doi.org/10.2172/759450>)
- [36] Roache P J 2002 Code verification by the method of manufactured solutions *Trans. ASME* **124** 4–10
- [37] Wilson H 2006 Neoclassical tearing modes *Fusion Sci. Technol.* **49** 155–63
- [38] Fischer R *et al* 2010 Integrated data analysis of profile diagnostics at ASDEX Upgrade *Fusion Sci. Technol.* **58** 675–84
- [39] Belli E A and Candy J 2008 Kinetic calculation of neoclassical transport including self-consistent electron and impurity dynamics *Plasma Phys. Control. Fusion* **50** 095010
- [40] Belli E A and Candy J 2012 Full linearized Fokker–Planck collisions in neoclassical transport simulations *Plasma Phys. Control. Fusion* **54** 015015
- [41] Peeters A *et al* 2009 The nonlinear gyro-kinetic flux tube code {GKW} *Comput. Phys. Commun.* **180** 2650–72
- [42] Pankin A *et al* 2004 The tokamak Monte Carlo fast ion module NUBEAM in the NationalTransport Code Collaboration library *Comput. Phys. Commun.* **159** 157–84
- [43] Angioni C *et al* 2011 Gyrokinetic modelling of electron and boron density profiles of H-mode plasmas in ASDEX Upgrade *Nucl. Fusion* **51** 023006
- [44] Nordman H *et al* 2011 Fluid and gyrokinetic simulations of impurity transport at JET *Plasma Phys. Control. Fusion* **53** 105005
- [45] Manas P *et al* 2017 Gyrokinetic modeling of impurity peaking in JET H-mode plasmas *Phys. Plasmas* **24** 062511
- [46] Bonanomi N *et al* 2018 Light impurity transport in JET ILW L-mode plasmas *Nucl. Fusion* **58** 036009
- [47] Casson F *et al* 2013 Validation of gyrokinetic modelling of light impurity transport including rotation in asdex upgrade *Nucl. Fusion* **53** 063026